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Title: On-the-Fly Nuclear Data Processing Methods for Monte Carlo Simulations

of Fast Spectrum Systems

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On-the-Fly Nuclear Data Processing Methods for Monte Carlo Simulations of Fast Spectrum Systems

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XCP-3 Group Meeting August 19, 2015

Jon Walsh August 19, 2015



Section 1

Unresolved Resonance Region

Jon Walsh



Guiding Objectives

High-fidelity, precise Monte Carlo simulations of realistic intermediate and fast spectrum systems

- Improved physics
 - Representation of nuclear data
 - Processing methods



Guiding Objectives

High-fidelity, precise Monte Carlo simulations of realistic intermediate and fast spectrum systems

- Improved physics
 - Representation of nuclear data
 - Processing methods
- Computational efficiency
 - Memory requirements
 - Calculation speed



Unresolved Resonance Region

- Intermediate-energy resonances can be experimentally unresolvable
- Nuclear data evaluators give us average information about cross section behavior in the URR
- Associated theoretical statistical distributions are known
- In nature, fine structure still exists

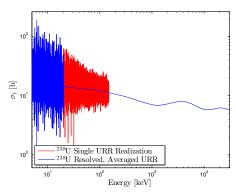


Figure: 238 U total cross section at 293.6 K



Averaged Pointwise Cross Sections

 A known, pointwise cross section is just a collapsed distribution,

$$\sigma_{\mathsf{x}}(\mathsf{E}) = \langle \sigma_{\mathsf{x}}(\mathsf{E}) \rangle = \int_{-\infty}^{\infty} d\sigma_{\mathsf{x}}' \delta(\sigma_{\mathsf{x}}' - \sigma_{\mathsf{x}}(\mathsf{E})) \sigma_{\mathsf{x}}' \tag{1}$$

• More generally, we can't collapse (resolve) the distribution to a point, so we have a Lebesgue integral in σ'_x -space,

$$\langle \sigma_{\mathsf{x}}(\mathsf{E}) \rangle = \int_{-\infty}^{\infty} d\sigma_{\mathsf{x}}' P(\sigma_{\mathsf{x}}'|\mathsf{E}) \sigma_{\mathsf{x}}' \tag{2}$$

This is what we have to deal with in the URR

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OTF Nuclear Data Methods



URR Self-Shielding Effects

- Averaging process is equivalent to generation of infinite-dilute cross sections
- No resonance structure
 - \rightarrow No flux perturbation
 - ightarrow No energy self-shielding

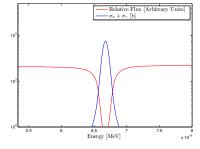


Figure: Energy self-shielding schematic



URR Self-Shielding Effects

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- Mis-predicted reaction rates
- Artificially high resonance absorption
- Under-predicted k_{eff} eigenvalues
- Unconservative critical assembly calculations

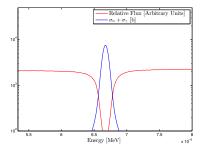


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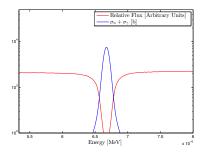


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 Resonance overlap can exacerbate the effect

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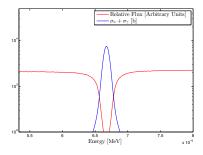
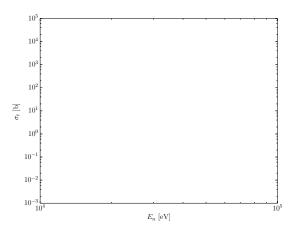


Figure: Energy self-shielding schematic

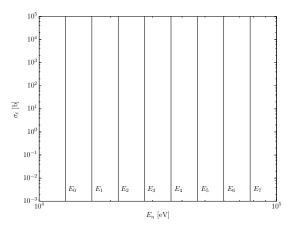
- Resonance overlap can exacerbate the effect
- Fine structure in the URR needs to be represented





• Need σ_t magnitudes as a function of E_n and T

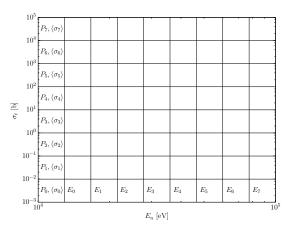




• At each discrete T, set an E_n mesh

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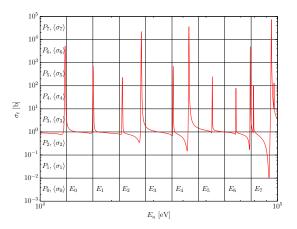
Probability Table Method



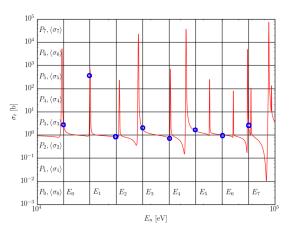
• At each discrete E_n , set a σ_t mesh

IIIiT

Probability Table Method



Randomly generate an independent realization of resonance structure



- Record band index and magnitude at each E_n
- Average over all samples to compute band probability and σ_t



- Stochastically generated tables of cross section values with associated probabilities of being realized at discrete energies AND temperatures
- Cross section values, $\{\hat{\sigma}_{x}^{1}(E_{i}), \hat{\sigma}_{x}^{2}(E_{i}), ..., \hat{\sigma}_{x}^{J}(E_{i})\}$, are sampled according to their associated probabilities, $\{\hat{P}_{t}^{1}(E_{i}), \hat{P}_{t}^{2}(E_{i}), ..., \hat{P}_{t}^{J}(E_{i})\}$, as necessary, throughout the transport simulation



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- Well-established method for treating URR cross section resonance structure
- Implemented in several nuclear data pre-processing codes (NJOY, PREPRO, PROTAB/RACER, CALENDF, AMPX, and others)



- Stochastically generated tables of cross section values with associated probabilities of being realized at discrete energies **AND** temperatures
- Cross section values, $\{\hat{\sigma}_{\mathbf{v}}^{1}(E_{i}), \hat{\sigma}_{\mathbf{v}}^{2}(E_{i}), ..., \hat{\sigma}_{\mathbf{v}}^{J}(E_{i})\}$, are sampled according to their associated probabilities, $\{\hat{P}_t^1(E_i), \hat{P}_t^2(E_i), ..., \hat{P}_t^J(E_i)\}$, as necessary, throughout the transport simulation
- Well-established method for treating URR cross section resonance structure
- Implemented in several nuclear data pre-processing codes (NJOY, PREPRO, PROTAB/RACER, CALENDF, AMPX, and others)
- Drawbacks include opaqueness of the generation process, the need for sensitivity/mesh refinement studies, and increased memory requirements



Section 2

On-the-Fly Cross Sections

Jon Walsh



Average resonance parameter values given for energy ranges

Jon Walsh



- Average resonance parameter values given for energy ranges
- Sample the theoretical distributions of those parameters

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- Continuous phase-space (E_n, T, σ) analog to probability tables



- Average resonance parameter values given for energy ranges
- Sample the theoretical distributions of those parameters
- Generate energy-localized resonance realizations (i.e. level spacings and partial widths) at each event
- Compute temperature-dependent SLBW resonance cross sections via $\psi-\chi$ Doppler integrals
- Continuous phase-space (E_n, T, σ) analog to probability tables
- Proceeds directly from temperature-independent resonance parameters

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Level Spacings

"Wigner's surmise" for distribution of level spacings:

$$P_{W}\left(\frac{D_{I,J_{j}}}{\langle D_{I,J_{j}}(E_{n})\rangle}\right) = \frac{\pi D_{I,J_{j}}}{2\langle D_{I,J_{j}}(E_{n})\rangle} \exp\left(-\frac{\pi D_{I,J_{j}}^{2}}{4\langle D_{I,J_{j}}(E_{n})\rangle^{2}}\right)$$
(3)

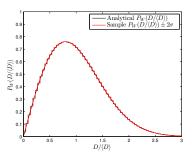


Figure: Sampled and analytical level spacing distributions

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Partial Widths

Partial reaction widths, Γ_r , are obtained by sampling a χ^2 distribution:

$$P_{\chi^{2}(\mu_{r})}(y) = \frac{\exp\left(-\frac{y}{2}\right)y^{\frac{\mu_{r}}{2}-1}}{2^{\mu_{r}/2}G\left(\frac{\mu_{r}}{2}\right)}; \qquad y \equiv \mu_{r}\frac{\Gamma_{r}^{I,J}}{\langle \Gamma_{r}^{I,J}(E_{n})\rangle} \tag{4}$$

Construction of a discrete distribution with N equiprobable bins:

$$\int_{y_{i-1}}^{y_i} P_{\chi^2(\mu_r)}(y') dy' = \frac{1}{N}; \quad i = 1, 2, ..., N; \quad y_0 = 0; \quad y_N \to \infty$$
(5)

$$\langle y' \rangle_i = N \int_{\gamma_{i-1}}^{\gamma_i} y' P_{\chi^2(\mu_r)}(y') dy' \tag{6}$$

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Single-Level Breit-Wigner Formulae

Elastic scattering:

$$\sigma_{n}(E_{n}) = \sum_{l=0}^{NLS-1} \sum_{j=1}^{NJS_{l}} \sum_{\lambda=1}^{N_{res}} \sigma_{\lambda} \left(\left[\cos \left(2\phi_{l}(\rho) \right) - \left(1 - \frac{\Gamma_{n,\lambda}}{\Gamma_{\lambda}} \right) \right] \psi(\theta, x) + \chi(\theta, x) \sin \left(2\phi_{l}(\rho) \right) \right) + \frac{4\pi}{k(E_{n})^{2}} \sum_{l=0}^{NLS-1} \left(2l+1 \right) \sin^{2} \left(\phi_{l}(\rho) \right)$$

$$(7)$$

· Reaction:

$$\sigma_r(E_n) = \sum_{l=0}^{NLS-1} \sum_{j=1}^{NJS_l} \sum_{\lambda=1}^{N_{\text{res}}} \sigma_{\lambda} \frac{\Gamma_{r,\lambda}}{\Gamma_{\lambda}} \psi(\theta, x)$$
 (8)

• $\psi - \chi$ Doppler integrals:

$$\psi(\theta, x) = \frac{\theta\sqrt{\pi}}{2} \operatorname{Re}\left[W\left(\frac{\theta x}{2}, \frac{\theta}{2}\right)\right]; \qquad \chi(\theta, x) = \frac{\theta\sqrt{\pi}}{2} \operatorname{Im}\left[W\left(\frac{\theta x}{2}, \frac{\theta}{2}\right)\right]$$
(9)

Faddeeva function:

$$W(\alpha, \beta) = \exp(-z^2)\operatorname{erfc}(-iz) = \frac{i}{\pi} \int_{-\infty}^{\infty} dt \frac{\exp(-t^2)}{z - t}$$
 (10)



Event-Based Cross Section Realizations

An on-the-fly URR cross section calculation capability is implemented in OpenMC

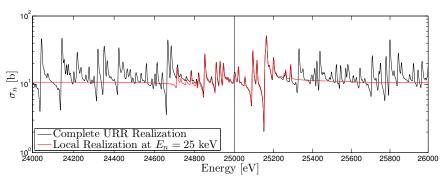


Figure: 238 U elastic scattering cross section realization at 25 keV

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SLBW: Verification

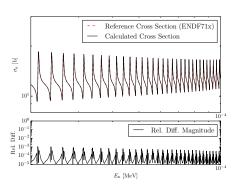


Figure : 239 U 293.6 K elastic scattering cross section

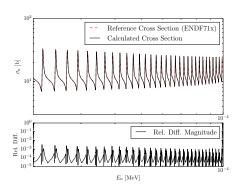


Figure: ²⁴³Pu 293.6 K elastic scattering cross section

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Mir

ZEBRA Critical Assembly



- Zero-Energy Breeder Reactor Assembly
- UK Atomic Energy Authority
- Fast reactor assemblies
- Significant URR effects (~1000 pcm)

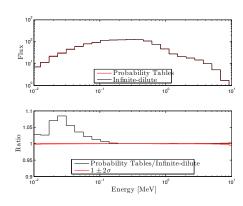


Figure: Infinite-dilute and resonance cross section flux spectra



On-the-Fly vs. Probability Tables: k_{∞}

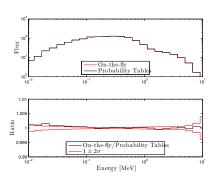


Figure: Comparison of on-the-fly and probability table flux spectra

k_{∞}	1σ
1.00914	0.00005
1.01897	0.00004
1.01892	0.00004
	1.00914 1.01897

 $\mbox{ Table : ZEBRA } k_{\infty} \mbox{ comparison for various URR } \\ \mbox{ treatments }$

- OTF eliminates the need for pre-processing and storage of temperature-dependent probability table data
- $\sim 1000X$ memory reduction
- $\sim 0.1 10X$ particle simulation rate slowdown

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Temperature-Dependent Probability Tables

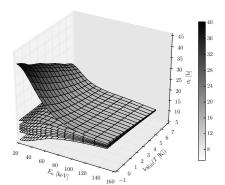


Figure: Equiprobable cross section magnitude surfaces

- Generate equiprobable cross section magnitude surfaces on an $E_n - T$ mesh
- Randomly sample magnitude and interpolate on mesh
- Compromise between typical probability table and fully OTF procedures



Temperature-Dependent Probability Tables

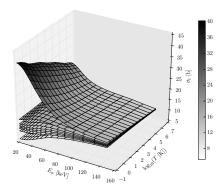


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- Randomly sample magnitude and interpolate on mesh
- Compromise between typical probability table and fully OTF procedures

ΔT [K]	$\textit{k}_{ ext{eff}}$	1σ
0	1.00466	0.00010
100	1.00463	0.00010
200	1.00468	0.00010
400	1.00533	0.00010

Table: Big Ten k_{eff} for various URR treatments



Section 3

Alternate Cross Section Representations

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- ENDF-6 format specifies use of SLBW for the URR
- Level-level interference is neglected
- Negative elastic scattering cross sections are possible in the resonance dips
- Multi-level Breit-Wigner (MLBW) capability implemented in OpenMC (probability tables, on-the-fly, pointwise)

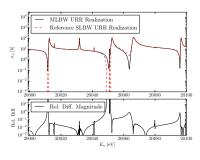


Figure : Comparison of SLBW and MLBW elastic scattering cross sections



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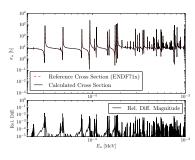


Figure: ²³⁴U 293.6 K elastic scattering cross section



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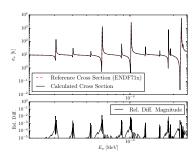


Figure : ^{244}Pu 293.6 K elastic scattering cross section



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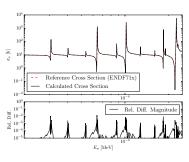


Figure: ²⁴⁴Pu 293.6 K elastic scattering cross section

Formalism	$k_{ m eff}$	1σ
SLBW	1.00461	0.00010
MLBW	1.00453	0.00009

Table : Big Ten $k_{\rm eff}$ for various URR treatments, 293.6 K



- Calculation of True expected values requires independent simulations, each using a single, independent resonance structure
- Pointwise URR cross section reconstruction capability implemented in OpenMC

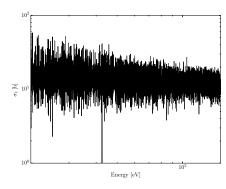


Figure: 238 U total cross section at 293.6 K



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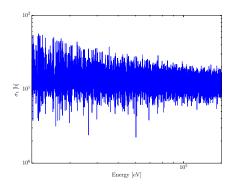


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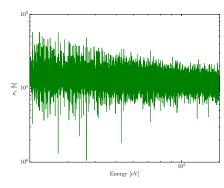


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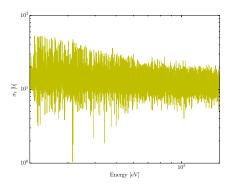


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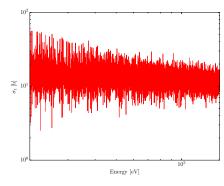


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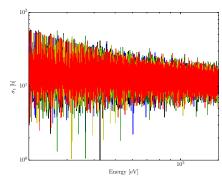


Figure: 238 U total cross section at 293.6 K



- Calculation of True expected values requires independent simulations, each using a single, independent resonance structure
- Pointwise URR cross section reconstruction capability implemented in OpenMC
- Probability tables reproduce expected values remarkably well (not required by theory)
- Spread of individual variates can be significant, 100's pcm on k_{eff}

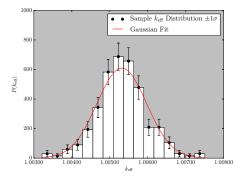


Figure : Distribution of Big Ten $k_{\rm eff}$ from independent URR realizations



Extended URR Evaluations

 Fast spectrum system fluxes often peak above the URR

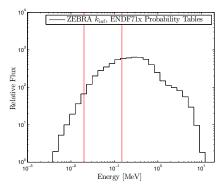


Figure: ZEBRA flux spectrum w/ ²³⁸U URR highlighted

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Extended URR Evaluations

- Fast spectrum system fluxes often peak above the URR
- There can still be resonance structure up there

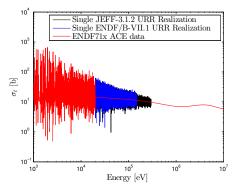


Figure: RRR and URR ²³⁸U total cross sections, 293.6 K



Extended URR Evaluations

- Fast spectrum system fluxes often peak above the URR
- There can still be resonance structure up there
- Neglect of resonance structure can strongly affect integral tallies
- Are there any evaluators in the room?!

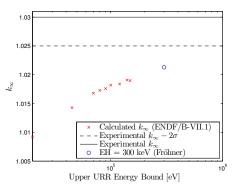


Figure : ZEBRA k_{∞} variation with EH



Competitive Reaction Resonance Structure

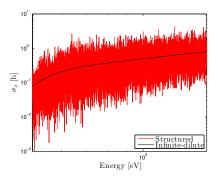


Figure: ²³⁸U first-level inelastic scattering resonance structure

- Competitive reactions

 (e.g. level inelastic
 scattering) have resonance
 structure
- ENDF-6 format allows a single competitive width
- Resonance structure cannot be accounted for properly if two competitive channels are open
- In practice, ALL competitive resonance structure is often neglected



Competitive Reaction Resonance Structure

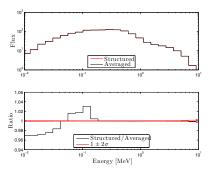


Figure: Flux spectra comparison

System	Competitive Cross Section	$k_{ m eff}$	1σ
Big Ten	Averaged	1.00459	0.00004
Big Ten	Resonant	1.00524	0.00005
ZEBRA	Averaged	1.01892	0.00004
ZEBRA	Resonant	1.02054	0.00004

Table : $k_{\rm eff}$ comparison for averaged and structured competitive cross sections, 293.6 K

- Competitive reactions (e.g. level inelastic scattering)
 have resonance structure
- ENDF-6 format allows a single competitive width
- Resonance structure cannot be accounted for properly if two competitive channels are open
- In practice, ALL competitive resonance structure is often neglected



Section 4

Doppler Broadening Secondary Distributions

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Motivation

Methods for Processing ENDF/B-VII with NJOY (my emphasis)

"The code uses the input value thnmax, or the upper limit of the resolved-resonance energy range, or the lowest threshold (typically>100 keV) as a breakpoint. No Doppler broadening or energy-grid reconstruction is performed above that energy. No broadening of thresholds is normally done, because we dont have methods to calculate the scattering distributions from broadened thresholds. There is an option to override this for applications like astrophysics that might desire to compute reaction rates for broadened thresholds."

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Motivation

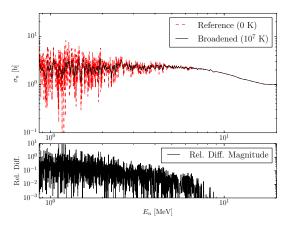
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 Doppler broadening of fast energy region cross sections – and reaction kernels, in general – is being restricted by an inability to broaden secondary distributions consistently



Doppler Broadened Fast Region Cross Sections



 $\textbf{Figure}: \ \text{Comparison of broadened} \ ^{56} \text{Fe elastic scattering cross section with 0 K reference}$



Reaction Kernel Broadening

 Typical formulation of Doppler broadening rigorously preserves integrated reaction rate for reaction x:

$$v\sigma_{x}(T,v) = \int_{\forall \vec{v}_{t}} d\vec{v}_{t} V(T, \vec{v}_{t}) v_{\text{rel}} \sigma_{x}(0, v_{\text{rel}})$$
 (11)

- Note: it is common to factorize $V(\mathcal{T}, \vec{v}_t)$ into independent distributions for μ_{in} and $|\vec{v}_t|$, as in the Maxwell-Boltzmann ideal gas model
- The effective, temperature-dependent cross section, $\sigma_x(T, v)$, is used to determine where in phase-space a reaction of type x occurs



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 (11)

- Note: it is common to factorize $V(\mathcal{T}, \vec{v}_t)$ into independent distributions for $\mu_{\rm in}$ and $|\vec{v}_t|$, as in the Maxwell-Boltzmann ideal gas model
- The effective, temperature-dependent cross section, $\sigma_x(T, v)$, is used to determine where in phase-space a reaction of type x occurs
- But, we also need to know the differential nature of the individual reaction events that occur (e.g. \vec{v}_t , $\mu_{\rm out}$)



Secondary Angular Distributions

- Nuclear data evaluations provide 0 K secondary angular distributions (sometimes implicitly via Legendre coefficients)
- It is common practice to independently sample these 0 K distributions
- Implicit assumption that distributions do not have significant energy dependence

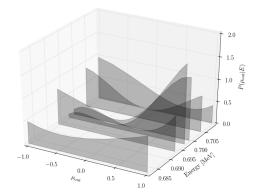


Figure: ⁵⁶Fe elastic scattering angular distribution



Broadening Angular Distributions

• Without loss of generality, the $\mu_{\rm out}$ -integrated cross section, $\sigma_x(0, v_{\rm rel})$, can be expanded into its angular components

$$v\sigma_{x}(T, v) = \int_{\forall \vec{v}_{t}} d\vec{v}_{t} \int_{-1}^{1} d\mu_{\text{out}} V(T, \vec{v}_{t}) v_{\text{rel}} P(\mu_{\text{out}} | v_{\text{rel}}) \sigma_{x}(0, v_{\text{rel}})$$
(12)

 Integrand and a normalization constant constitute, by definition, a probability density function for the consistent,
 Doppler broadened double-differential reaction kernel

$$P(\vec{v}_t, \mu_{\text{out}}|v) = \frac{1}{v\sigma_x(T, v)} V(T, \vec{v}_t) v_{\text{rel}} P(\mu_{\text{out}}|v_{\text{rel}}) \sigma_x(0, v_{\text{rel}})$$
(13)

Jon Walsh

August 19, 2015

OTF Nuclear Data Methods



0 K and Broadened Angular Distributions

• Integrate over \vec{v}_t to obtain the broadened scattering cosine distribution and compare with the 0 K data

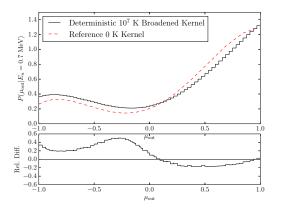


Figure: Comparison of broadened kernel with 0 K reference



0 K and Broadened Angular Distributions

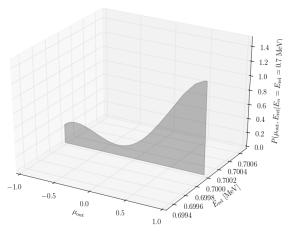


Figure: Comparison of broadened kernel with 0 K reference



0 K and Broadened Angular Distributions

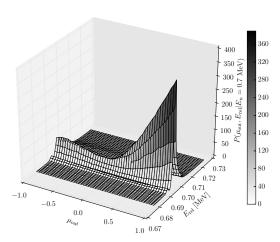


Figure: Comparison of broadened kernel with 0 K reference



Reaction Kernel Broadening Methods

- $P(\vec{v}_t, \mu_{\text{out}}|v)$ can be deterministically Doppler broadened, as was previously shown
- Broadened kernels can then be straightforwardly sampled in Monte Carlo simulations
- Such a treatment is perfectly legitimate and an improvement over the state of the practice
- However, the kernels are only exact at the precise temperature to which they are broadened
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- However, the kernels are only exact at the precise temperature to which they are broadened
- What about multiphysics? Possibly dramatic increase in secondary distribution memory requirements
- An entirely equivalent stochastic sampling method is derived and implemented in OpenMC
- No memory requirement penalty incurred 0 K data only



Verification

Excellent agreement between stochastic and deterministic kernels

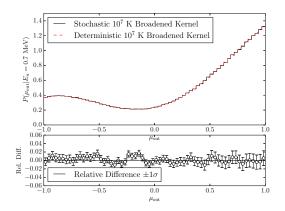


Figure: Comparison of stochastic and deterministic broadened kernels



Integral Simulations

- The stochastic kernel broadening method has been implemented for elastic scattering in the continuous-energy neutron transport code OpenMC
- Preliminary studies of integral effects are underway
 - ²³⁹Pu sphere reflected by ⁵⁶Fe
 - Negligible bias at room temperature at the few-pcm level
 - \bullet $+609\pm10$ pcm bias at 10^7 K
- Suggestions for systems in which this effect is important?
- Design criteria:
 - High-temperatures to get secondary distributions that change over the range of attainable relative energies
 - Fast spectra typically needed to reach structured (i.e. non-isotropic) scattering distributions
 - ⁵⁶Fe, ⁹Be, C, possibly others



Conclusions

 A flexible probability table interpolation scheme has been implemented and tested with results comparing favorably to the continuous phase-space on-the-fly approach



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- A flexible probability table interpolation scheme has been implemented and tested with results comparing favorably to the continuous phase-space on-the-fly approach
- Several alternate resonance data representations have been implemented and tested with results showing varying degrees of significance
- A fully consistent, memory-efficient stochastic method for Doppler broadening of the double-differential elastic scattering kernel has been derived, implemented, and verified



Future Work

- Continued analyses with the presented methods
- Extend Doppler broadening of reaction kernels
 - Secondary angular distributions for other reactions
 - Secondary energy distributions
 - Correlated angle-energy distributions
- Validation of double-differential broadening?
- Extended URR evaluation?
- Graduate?



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